Data Source: **EM CDB** Report Number: GEN-01b

Operations/Field Office: Savannah River Print Date: 3/9/2000

HQ ID: 0495 Site Summary Level: Savannah River Site

Project SR-SF02 / L Area Spent Nuclear Fuel Project

### **General Project Information**

### **Project Description Narratives**

#### Purpose, Scope, and Technical Approach:

In 1992, the Secretary of Energy directed the Assistant Secretary for Environmental Restoration and Waste Management (EM) to develop an integrated, long-term Spent Nuclear Fuel (SNF) management program. In response, EM initiated the development of a DOE-owned SNF program to define and ensure resolution of all associated issues starting with the quantification of DOE SNF inventories and fuel storage facilities. The purpose of the DOE-owned SNF program is to integrate DOE's existing SNF activities into one program to better control and manage this material and to ensure that all issues associated with SNF are resolved in a safe and cost effective manner. The intent is to resolve the remaining vulnerabilities of existing wet storage of SNF to minimize health and safety risks. WSRC will continue to provide a central point of contact for DOE-SR for planning and coordination of all off-site shipments of DRR/DOE/FRR SNF into SRS Basins. After the startup of the new Treatment & Storage Facility (TSF) in 105-L, WSRC will continue planning and integration of shipments of DOE SNF into the new TSF, via L-Basin.

The Nuclear Weapons Nonproliferation Policy Concerning Foreign Research Reactor Spent Nuclear Fuel Environmental Impact Statement Record of Decision was issued on May 13, 1996. Key impacts from this document include a provision to return about 17,800 FRR SNF elements to SRS, including L, between 1996 and 2009, although that number has since been reduced due to some foreign countries electing not to participate as originally thought.

The Programmatic Spent Nuclear Fuel Management and Idaho National Engineering Laboratory Environmental Restoration and Waste Management Programs final EIS ROD was issued on May 30, 1995. Key impacts from this document include a provision to consolidate aluminum clad SNF at SRS by means of shipping FRR SNF elements, as well as those from US Universities, other DOE sites, and other US Government sites. L plays a crucial role in this ROD vis-à-vis fuel receipts.

SNF will be received at SRS from off-site domestic research reactors (DRR), government, and foreign research reactor (FRR) sources. More than 1400 casks are currently expected, over the project lifetime, with foreign sources accounting for approximately 500 of the total. The receipts are expected to average about 60 casks per year between 1999 and 2009. The receipt rate will vary after 2009 consistent with the availability of the SNF. Receipts could continue until 2035, pending the availability of the federal repository to take receipts directly. Any projected receipts are for planning purposes only, and may change due to numerous conditions beyond the control of DOE or the M&O contractor. However, minor changes will not affect budget requirements.

The program utilizes L-Reactor Basin as the primary receipt and storage facility for off-site SNF.

L-Area contains the 105-L Reactor Building (1,000,000 square feet) along with support facilities. Inside the 105-L Reactor Building is the Disassembly Basin, which stores SRS SNF and will be used to store SNF from off-site Research Reactors. Off-site SNF will be received in L Basin even after the new TSF is fully operational. The L-Area disassembly area includes equipment to control basin water quality for the storage of SNF. The basin walls are below ground and are constructed of thick steel-reinforced concrete. The basin contains approximately 3.5 million gallons of water and varies in depth from 17 to 30 feet.

The scope of this project includes all programmatic and physical support efforts related to safe receipt and storage of SNF; 941 shipments of

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### **Project Description Narratives**

irradiated fuel to H-Canyon; and stabilization required to maintain L-Reactor in the cold shutdown condition with no capability of restart. Basin Ops and S&M activities will continue in L-Area with a four-shift operation. With the assumption that the Treatment and Storage Facility will be located in the 105-L building, L-Disassembly Basin will receive off-site cask shipments, unload the casks, inspect, and prepare fuel for storage. Disassembly Basin Upgrade de-ionization system for L-Basin will be operated in support of fuel storage requirements. Assuming H-Canyon operates, Mk16 SNF in L-Basin will be transferred to H-Canyon for processing beginning in 2QFY01, following K-Basin deinventory. Per the latest Phased Canyon Strategy (dated 2/4/99), L-Basin deinventory of 94-1 SNF is scheduled to be completed by the end of 20FY02. SNF will be transferred from L-Basin to the TSF when it is ready for operation.

In order to accommodate the off-site and RBOF fuel prior to operation of the new Treatment and Storage Facility, additional racks are required in the basin. These additional racks are scheduled to be added as sub-projects of the Disassembly Basin Upgrades Line Item (95-D-158) to allow for the required additional storage capacity of spent nuclear fuel at SRS. These sub-projects will allow for design, fabrication, and installation of additional (phase II) High Flux Isotope Reactor (HFIR) racks; design, fabrication, and installation of additional (phase II) Materials Test Reactor (MTR) racks; and design, fabrication, and installation of Stainless Steel/Zircaloy (SS/Zr) clad fuel racks. (See DBU PBS for more detail.) In the case of the SS/Zr clad fuel, preliminary SFSD engineering evaluation indicates multiple rack designs are needed for the basin to accommodate the assortment of fuel assembly and container shapes. Few details are available on composition of the individual items. The key upfront activity, necessary for successful completion of the SS/Zr-clad fuel transfer, is development of a conceptual plan with details for the disposition of the material. Attention must be focused on defining the scope of characterization required prior to shipment off-site to either Idaho or the geologic repository. If the racks for SS/Zrclad SNF are to be located in L-Basin, the disposition of 189 bundles of Mk16B's and removal of scrap from L-Basin must be completed to provide access for rack installation to accommodate the SS/Zr-clad materials from RBOF. This results in the earliest possible transfer of SS/Zr-clad material from RBOF to L-Basin in 3/02. If H-Canyon does not operate, K-basin could be utilized for expediting the SS/Zr-clad fuel deinventory. The assumption is that the 2-5 man-years of up-front engineering effort to develop detailed planning to support the SS/Zr-clad fuel transfers is completed consistent with the windows of opportunity in each case.

As L-Area is in large part radioactively contaminated, basic surveillance and maintenance activities must be performed to ensure that 1) the facility continues to pose acceptable risk to the environment, site workers, and the general public; 2) the facility is maintained in accordance with the Authorization Basis requirements; and 3) the facility continues cost effective management, planning, and oversight. Surveillance includes monitoring or observing activities required to ensure equipment/systems operate in compliance with the Authorization Basis such that the facility continues to pose acceptable risk to the environment, site workers, and the general public; and activities necessary for cost effective management, planning, and oversight. Maintenance includes day-to-day activities to preserve facilities and equipment from failure or decline to ensure they can perform their design function. Corrective and preventive maintenance for electrical, mechanical, hydraulic, and monitoring equipment and systems includes that associated with habitability, life safety, fire protection, ventilation, and environmental protection.

Included in the L-Area Project is the SRS Dam Restoration Program. Engineering analyses indicate that the Pond 2 and Pond B dams will require significant maintenance/repair in the near future. Pond 5 needs further evaluation. The expense associated with dam refurbishment leads to the need to determine whether the dams are needed or whether the dams (and associated ponds) can be eliminated. The issue is complicated by the fact that both ponds are potentially contaminated. Outvear funding requirements would be determined by the outcome of the evaluation and NEPA processes. Minimal S&M activities and dam maintenance will keep L-Lake in a standby mode pending a decision to shut it down in the future.

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Project SR-SF02 / L Area Spent Nuclear Fuel Project

#### **Project Description Narratives**

L-Area provides for the consolidated storage of heavy water and its associated surveillance and maintenance. This consolidation reduces site-operating expenses by allowing the shutdown of systems in remote areas (ventilation and heating) required only for heavy water storage. SFSD Conduct of Operations Improvements necessary to achieve site safety standards may require alternative funding including some program sacrifices in FY00 and FY01 in order to pay for these items. These include Configuration Management, Linking Document, Increased Rad Con, and the Asset Information Management system (AIMS).

#### **Project Status in FY 2006:**

Deinventory of the L Disassembly Basin of offsite fuel will start in FY05 and will be in progress during FY06, assuming that the TSF is on line by the end of FY05. Basin Management and S&M activities described above will continue. Assembly throughput to TSF is planned until deinventory is complete. Off-site fuel receipts/RBOF deinventory into L-Basin will continue during 2006.

#### Post-2006 Project Scope:

SNF stored in L-Basin will be processed in the TSF facility starting in FY05 and will continue through about FY35. Demobilization activities will start after the completion of basin deinventory at which time L-Reactor will be placed in a low cost S&M mode.

It should be noted that RBOF operates a resin regeneration facility for deionization of resins from RBOF and K/L disassembly basin water purification systems. Prior to RBOF shutdown (post 2006), an alternative for management of the Reactor deionization resins must be provided (new system in L-Area, once through and burial, procurement of portable skid-mounted unit, etc.)

L-Area will be turned over for decommissioning after stabilization. Decommissioning activities will be covered in separate ACP Projects (PBS SR-FA20 - Reactor Monitoring Projects and PBS SR-FA12 - L-Reactor Deactivation).

#### **Project End State**

L-Area Disassembly Basin will be deinventoried. Demobilization of the facility will be complete, followed by low cost S&M. Transfer for decommissioning will occur.

L-Area will be decommissioned to the extent necessary to meet the guidelines for a nuclear industrial zone under separate PtC Projects. (PBS SR-FA20 - Reactor Monitoring Projects and PBS SR-FA12 - L-Reactor Deactivation).

#### **Cost Baseline Comments:**

Note: In the L-Area SNF Project scope description, WSRC assumes a receipt of \$3M (burdened) from WNO2, Spent Fuel Receipts, to offset EM costs in FY00 and FY01. The funding levels shown on the PBS reflect that assumption.

Additionally, a top funding priority as indicated on the SRS site critical needs list is to restore \$3,700K to L S&M in FY00. These dollars were originally reduced from L S&M to help fund the Treatment and Storage Facility. This was intended to serve only as a budget placeholder. (Reference: Letter dated February 12, 1999; Greg Rudy to Ambrose Schwallie - Subject: Fiscal Year (FY) 2000 Budget)

The financial figures for the Path to Closure (PtC) were derived using the SRS FY99 Annual Operating Plan (AOP) as the beginning basis. Outyear budget (OYB) requirements were estimated by factoring Detailed Information Input Forms (DIIF's) and outyear program planning

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Project SR-SF02 / L Area Spent Nuclear Fuel Project

#### **Project Description Narratives**

assumptions/schedules against this AOP baseline. The OYB process utilizes the program requirements contained in the DOE Strategic Execution Guidance (SEG) as the formulation basis of detailed program/operating assumptions and Program Planning Packages used to communicate scope of work requirements to other SRS divisions, e.g. Construction, Waste Management, Environment, Safety & Health, etc. Financial estimates are generated by the line and support organizations using the DIIF system. Estimates were escalated for anticipated inflation using a 3.6% factor for FY00and 01, and 2.7% for FY02 and beyond - per the guidance from the site.

The full cost of PBS work scope may change based on the authorized funding and priorities in any given year due to changes in site overhead assumptions. For planning and budgeting purposes, work scope costs were estimated using site overhead rates sized for clearance at a funding target of \$1,222.5 million. For FY2001 (the budget year), the site overhead is applied and cleared at the funding target, while the work scope below the funding target (planning level) is incremental direct cost. For FY2002, the site overhead is applied and cleared over the total planning level of scope.

#### Safety & Health Hazards:

The L-Area facilities operations are performed in the following areas: the Disassembly Basin, the Stack Area, the -40 foot-elevation storage room housing the Moderator Storage Tanks and other areas containing drums of contaminated moderator, the LLW Compactor, and Process Room (Reactor Tank). The Disassembly Basin Area contains irradiated fuel and target assemblies and contaminated water. The Stack Area is where casks containing foreign and domestic research reactor fuel are removed from ISO or other shipping containers and later decontaminated after fuel has been removed. The contaminated moderator is stored in the Moderator Storage Tanks in the -40 foot-elevation storage room and other areas in drums. The LLW Compactor is in a fan room of Building 105-L. The Reactor Tank contains irradiated aluminum and stainless steel reactor components. The irradiated components are in solid physical form and their radioactive contaminates are fixed on irradiated components. Therefore, the irradiated aluminum and stainless steel components in the Reactor Tank do not pose a radioactive contamination release potential for the offsite public, but they do serve as a potential for onsite worker exposure.

The major hazards associated with L-Area result from continued storage of contaminated moderator, irradiated fissile materials, irradiated aluminum and stainless steel reactor components, unirradiated reactor fuel SNM, and storage of entrained radionuclides throughout 105-L, and within components and the storage of chemicals necessary for the continued operation of these activities.

L-Area is classified as a Hazard Category 2 facility. Hazard Category 2 facilities have the potential for significant on-site radiological or chemical consequences. The criteria for determining the radiological hazard categories are provided in DOE-STD-1027-92. Hazard Categorization and Accident Analysis Techniques for Compliance with DOE Order 5480.23, Nuclear Safety Analysis Report, and the criteria for determining the chemical hazard categorization are provided in WSRC-MS-92-206.

A hazards assessment for the L-Reactor Facility was performed to determine its overall hazard categorization (Hazards Assessment Document for L-Reactor in Cold Shutdown (U), S-HAD-L-00001, Rev. 0). Radiological hazards of consequence were identified in the following areas: Disassembly Basin, areas containing contaminated moderator in tanks or drums, and LLW Compactor. The overall categorization is based upon the consequences of a postulated radionuclide and or/chemical release from the L-Reactor Facility, as well as the potential for criticality.

The criteria for determining the radiological hazard categories are provided in DOE-STD-1027-92, and the criteria for determining the chemical hazard categorization are provided in WSRC-MS-92-206. Determination of the above hazards are described in WSRC-TR-95-0054, Rev. 0, "Basis For Interim Operation (BIO) For The L-Reactor Facility (U)," Section 6, "Approach For Hazard Identification/Categorization." Chemical inventory is controlled in accordance with RDP 14.1 "Chemical Management Program," and "Chemicals and Non-radioactive Hazardous Materials Control (U), DPSOL 105-1845-K."

Hazards present in the L-Reactor Area will vary during the accelerated cleanup of the facility. Basin operations will continue in L-Area until

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### **Project Description Narratives**

deinventory of irradiated nuclear materials is completed. Surveillance and maintenance activities continue through the demobilization/deinventory stages, at which point FDD will assume custodianship of the facility. The demobilization plan will contain provisions for ensuring adequate surveillance and maintenance is performed during transition phase. Surveillance and maintenance requirements that will be addressed are: radiation protection, hazardous chemical safety, physical safety and security, and potential public and environmental hazards.

#### Safety & Health Work Performance:

As described in DOE's, "Safety Management System Policy," P450.4, there are six primary components that must be implemented; Objective, Principles, Functions, Implementation, Responsibilities, and Mechanisms. In adopting these components into the WSRC program, WSRC developed the Safety Management System Policy MP1.22, "Integrated Standards Based Safety Management Program," and submitted to the DOE (WSRC letter ESH-97-0004, F. B. Davis to L. C. Sjostrom, "Schedules for Implementation of a Safety Management System (SMS) (U)," dated March 17, 1997) WSRC-IM-97-10, Rev.0, "Safety Management System Description (U). These documents describe the Safety Management System used to ensure safety is integrated into work performed under WSRC's Contract No. DE-AC09-96SR1850.

The Department of Energy has determined (Authorization Agreement for L-Reactor Facility, 5480.23.16-L-Area-AA, Revision 0) through a series of comprehensive reviews, that L-Reactor Facility will be operated in compliance with the Standards/Requirements Identification Document (S/RID).

The Basis for Interim Operation for the L-Reactor Facility (WSRC-TR-95-0054, Revision 0, as amended) documents the analysis of the facility hazards, specifies the controls necessary to prevent and mitigate the hazards, and defines a safety management program which affords an acceptable level of safety to the public, the workers, and the environment.

Through the performance of a Readiness Assessment and continuing operational assessments, there is reasonable assurance the facility can be operated without endangering the health and safety of the public, the workers, or the environment.

Activities and checkpoints are described by the Integrated Management System Description. The conditions and requirements are clearly established and agreed upon prior to the starting of any project and those requirements are contractually binding upon WSRC. The key elements of the WSRC Integrated Safety Program are to define the scope of work, identify and analyze hazards associated with the work, develop and implement hazard controls, perform work within controls, and provide feedback on adequacy of controls and continue to improve safety management. The WSRC Integrated Procedures Management System is the primary mechanism for implementing the objective, principles and functions of the Safety Management System. This system establishes Company-Level, Division-level, and Program-specific procedures consistent with organizational roles, and ensures a consistent, discipline site-wide approach to safety while performing work.

#### **PBS Comments:**

The Nuclear Weapons Nonproliferation Policy Concerning Foreign Research Reactor Spent Nuclear Fuel Environmental Impact Statement Record of Decision was issued on May 13, 1996. Key impacts from this document include a provision to return about 17,800 FRR SNF elements to SRS, including L, between 1996 and 2009, although that number has since been reduced due to some foreign countries electing not to participate as originally thought.

The Programmatic Spent Nuclear Fuel Management and Idaho National Engineering Laboratory Environmental Restoration and Waste Management Programs final EIS ROD was issued on May 30, 1995. Key impacts from this document include a provision to consolidate aluminum clad SNF at

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Project SR-SF02 / L Area Spent Nuclear Fuel Project

### **Project Description Narratives**

SRS by means of shipping FRR SNF elements, as well as those from US Universities, other DOE sites, and other US Government sites. L plays a crucial role in this ROD vis-à-vis fuel receipts.

A draft of the Phase I of the SRS SNF EIS was initiated in FY97; final issue will be during FY99.

#### **Baseline Validation Narrative:**

Independent validation efforts were done at SFSD in Fiscal Year 1998. These efforts were completed by an organization external to SFSD in a focused manner. The validation included a representative sample of programmatic operations, maintenance activities, and crosscutting overhead and service activities. As a result of this validation effort, the SFSD received the "validated" concurrence from the team.

#### General PBS Information

**Project Validated?** Yes Date Validated: 10/30/1998

Has Headquarters reviewed and approved project?

No

**Date Project was Added:** 12/1/1997 **Baseline Submission Date:** 7/3/1999

FEDPLAN Project? Yes

**Drivers: CERCLA** RCRA **DNFSB AEA** UMTRCA State **DOE Orders** Other Ν Ν Y Ν Ν Y Y Y

**Project Identification Information** 

**DOE Project Manager:** Sandra L. Johnson

**DOE Project Manager Phone Number:** 803-557-3828 **DOE Project Manager Fax Number:** 803-557-3996

DOE Project Manager e-mail address: sandra-l.johnson@srs.gov

Is this a High Visibility Project (Y/N):

#### **Planning Section**

**Baseline Costs (in thousands of dollars)** 

1997-2006 2007-2070 1997-2070 1997 1998 Actual 1999 2000 2001 2002 2003 2004 2005 2006 Actual Total Total Total 1997 1998

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Project SR-SF02 / L Area Spent Nuclear Fuel Project

Project SR-SF02 /	L Area S	Spent Nu	clear Fuel	Project												
PBS Baseline (current year dollars)	421,27	3 2,010,	736 2,4	32,009	23,090	23,090	20,833	20,833	35,724	36,187	43,055	48,434	52,327	53,711	53,377	54,535
PBS Baseline (constant 1999 dollars)	380,23	4 1,069,	.498 1,4	49,732	23,090	23,090	20,833	20,833	35,724	34,930	40,115	43,940	46,224	46,199	44,705	44,474
PBS EM Baseline (current year dollars)	421,27	3 2,010,	736 2,4	32,009	23,090	23,090	20,833	20,833	35,724	36,187	43,055	48,434	52,327	53,711	53,377	54,535
PBS EM Baseline (constant 1999 dollars)	380,23	4 1,069,	,498 1,4	49,732	23,090	23,090	20,833	20,833	35,724	34,930	40,115	43,940	46,224	46,199	44,705	44,474
	2007	2008	2009	2010	2011- 2015	2016- 2020	2021- 2025	2026- 2030	2031- 2035	2036- 2040	2041- 2045	2046- 2050	2051- 2055	2056- 2060	2061- 2065	2066- 2070
PBS Baseline (current year dollars)	52,554	53,973	55,430	56,926	308,534	275,261	308,534	352,497	402,724	144,303		) (	0 0	0	(	0
PBS Baseline (constant 1999 dollars)	41,731	41,731	41,731	41,731	208,952	163,168	160,083	160,083	160,083	3 50,205		) (	0 0	0	(	0
PBS EM Baseline (current year dollars)	52,554	53,973	55,430	56,926	308,534	275,261	308,534	352,497	402,724	144,303		) (	0 0	0	(	) (
PBS EM Baseline (constant 1999 dollars)	41,731	41,731	41,731	41,731	208,952	163,168	160,083	160,083	160,083	3 50,205		) (	0 0	0	(	) (
Baseline Escalation	Rates															
	1997	1998	1999	2000	200	01 2	2002	2003	2004	2005	2006	2007	2008	2009	ı	
	0.00%	0.00%	0.00%	3.60%	3.60	2.	70% 2	70%	2.70%	2.70%	2.70%	2.70%	2.70%	2.70%		
	2010	2011-2015	2016-2020	2021-202	5 2026-203	30 2031-2	2035 2036	-2040 204	1-2045 20	2046-2050	051-2055	2056-2060	2061-2065	2066-2070		

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Date of Dataset: 9/20/1999

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Project SR-SF02 / L Area Spent Nuclear Fuel Project

### **Project Reconciliation**

**Project Completion Date Changes:** 

Previously Projected End Date of Project: 9/1/2011

Current Projected End Date of Project: 9/30/2037

Explanation of Project Completion Date Difference (if applicable):

**Project Cost Estimates (in thousands of dollars)** 

Previously Estimated Lifecycle Cost (1997 - 2070, 1998 Dollars): 433,008 Actual 1997 Cost: 23,090 Actual 1998 Cost: 20,833

Previously Estimated Lifecycle Cost of Project (1999 - 2070, 1998 Dollars): 389,085 Inflation Adjustment (2.7% to convert 1998 to 1999 dollars): 10,505

Previously Estimated Lifecycle Cost (1999 - 2070, 1999 Dollars): 399,590

**Project Cost Changes** 

Cost Adjustments Reconciliation Narratives

**Cost Change Due to Scope Deletions (-):** 

**Cost Reductions Due to Efficiencies (-):** 

Cost Associated with New Scope (+): 1,006,216 The TSF will be located in 105-L and not privatized, facility life extended to 2037.

**Cost Growth Associated with Scope Previously Reported (+):** 

Cost Reductions Due to Science & Technology Efficiencies (-):

Subtotal: 1,405,806
Additional Amount to Reconcile (+): 3

Current Estimated Lifecycle Cost (1999 - 2070, 1999 Dollars): 1,405,809

Milestones

Milestone/Activity Field Milestone Original **Baseline** Legal **Forecast** Actual EA DNFSB Mgmt. Key Intersite Code Date Date Commit. Decision Date Date Date Basin deinventory SR-SF02-3 10/31/2010 Y

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Project SR-SF02 / L Area Spent Nuclear Fuel Project

Milestones													
Milestone/Activity	Field M Co		Original Date	Baseline Date	Legal Date	Forecast Date	Actual Date	EA	DNFSB	Mgmt. Commit.	Key Decision	Intersite	
Deinventory of 94-1 SNF to Cany	SR-SF02-2			12/31/2001					Y				
National/Tri-Party Decision, also SNF Proj, SF0	SR-SF02-1			12/1/1998		12/31/1999							
Project Mission Complete	SR-SF02-4			9/1/2011									
Project Start	SR-SF02-00	)1		10/1/1996									
Project Start		SR-SF01-00	)1	10/1/1996									
Complete L-Area 120 ton crane n	nodifications	SR-SF02-00	)2		9/30/2000								
Complete transfer of DNFSB 94-	SR-SF02-00	03		3/31/2002					Y				
Project Complete	SR-SF02-00	)4		9/30/2037									
Milestones - Part II													
Milestone/Activity	Field Milestone Code	Critical Decision	Critial Closure Pat	Project h Start	•	Missio Comple		Work Scope Risk	Intersite Risk	Cancel	led	Milestone D	Description
Basin deinventory	SR-SF02-3									Y			
Deinventory of 94-1 SNF to Canyon	SR-SF02-2									Y			
National/Tri-Party Decision, also impacts SF03-RBOF SNF Proj, SF0	SR-SF02-1									Y			
Project Mission Complete	Project Mission Complete SR-SF02-4									Y			
Project Start	SR-SF02-001			Y							date curre Depa start	for the planning ontly being emp ortment of Ener	ployed by the egy. The actual cility covered in
Project Start	SR-SF01-001			Y						Y		date represents for the planning	the beginning g document

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Project SR-SF02 / L Area Spent Nuclear Fuel Project

Milestones - Part II														
Milestone/Activity		Field Milestone Code	Critical Decision	Critial Closure Pat	Project h Start	Project End	Mission Complete		Work cope Risk	Intersite Risk	Cancelled		Milestone Des	cription
												Departn start dat	ly being employ ment of Energy te for the facilit S is in the early	. The actual ty covered in
Complete L-Area 120 ton crai modifications	ne S	SR-SF02-002											ete modification 20 ton crane and ions.	
Complete transfer of DNFSB materials	94-1 \$	SR-SF02-003										material	ete transfer of D ls from L Basir facilities.	
Project Complete	S	SR-SF02-004				Y								
Performance Measure	Metri	es												
Category/Subcategory	Units	1997-2006 20 Total	007-2070 19 Total	997-2070 Total	Actual Pre-1997	Planned 1997	Actual 1997	Planned 1998	Planned 1999				anned Plan 2002 2	nned Planne 2003 200
Tech.														
Deployed	Ntd	1.00	0.00	1.00					1.0	0				
Category/Subcategory	Units	Planned 2004	Planned 2005	Planned 2006	Planned 2007	Planned 2008	Planned 2009		0 20		2016 - 2020	Planned 2021 - 2025	Planned 2026 - 2030	Planned 2031 - 2035
Tech.														
Deployed	Ntd													
Category/Subcategory	Units	Planned 2036 - 2040	Planned 2041 - 2045	Planned 2046 - 2050	Planned 2051 - 2055	Planned 2056 - 2060	Planned 2061 - 2035	Planned 2066 2070	- '	ions Lif	ecycle Total			
Tech.														
Deployed	Ntd								-	1.00	1.00			

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Project SR-SF02 / L Area Spent Nuclear Fuel Project

### **Technology Deployments**

Deployment Year

Deployment Status Planned Forecast Actual Date

**Technology Name:** Prototype Tritium Air Monitor from standard commercially available components

Deployment Commitment 1999

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